Nuclear Reactor Fuel and Materials

Keyword: nuclear fuel, property, integrity

Uno Lab.

Masayoshi UNO, Dr. (Professor)

Property estimation and integrity prediction of nuclear fuels and materials in normal and accident conditions



Property measurement of simulated fuels Property esitimation by chemical calculations Development of measurement techniques

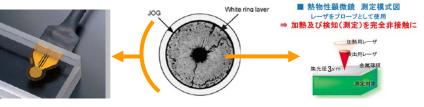
OUTLINE

Properties of pellets vary with burnup by FPs and metallurgical faults and Zr alloy claddings degrade by oxidation and hydrogen absorption in a reactor. Such variation of the properties are studied by experiments of simulated fuels and chemical calculations and the integrity of the fuel is estimated. Techniques for the property measurement of irradiated fuels in a hot cell are also developed.

RESEARCH WORKS

Development of techniques for thermal conductivity measurement of irradiated pellets.

Techniques to measure thermal conductivity of a whole pellet and that of the precipitates in a hot cell are developed using "hot desk method (HD)" and "thermal microscope (TM)", respectively.



Annular sensor and analysis based on 1-D thermal conductance model made to estimate thermal conductivity of annular pellets.

HD sensor

Irradiated fuels

Principle of TM

Annular HD sensor

25

20 15

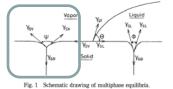
10

Measurement of contact angels of CeO₂ grain with CsI melt

In order to establish the thermodynamic equilibrium considering excess surface and interface energies in a debris contact angels of $CeO_2(UO_2)$ with CsI melt are measured.

$$G = G^B + G^S + G^I$$

Constrained Free Energy Minimization Method



SEM photo of

CeO₂ crystal

105 110 115 120 125 130 135 140 145

Measured contact angle

Measurement of interface energy

Reaction behavior of corium, sodium and concrete materials

To develop estimation methods for integrity of container vessel of sodium fast reactor in a severe accident condition the reaction behavior of corium, sodium and concrete materials is studied by experiments using simulated material including uranium as well as sodium tests and thermodynamic calculations.



X-ray diffraction

DSC

Furnace for sodium compounds

DSC

Laser flash

Fuel study; exp. of simulated materials, chemical calc. and exp. of actual fuel in a hot cell

Researches using Uranium and Thorium in the Lab. sometimes in collaboration with Osaka university and/or JAEA

Study on behavior during severe accidents and treatment of melted fuels

MESSAGES FOR APPLICANTS

FUTURE PERSPECTIVE

Students who study or want to study material science are welcomed. You will test Uranium and Thorium compounds and acquire calculation chemistry. You also have an opportunity for internship at JAEA and so on using more real materials.